



Agenzia nazionale per le nuove tecnologie, l'energia  
e lo sviluppo economico sostenibile



*Ministero dello Sviluppo Economico*

## RICERCA DI SISTEMA ELETTRICO

### Prosecuzione della Partecipazione a Comitati e Gruppi Internazionali

*F. Mascari, G. Vella*



RdS/2012/164

PROSECUZIONE DELLA PARTECIPAZIONE A COMITATI E GRUPPI INTERNAZIONALI

*F. Mascari, G. Vella CIRTEN*

Settembre 2012

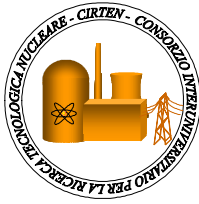
Report Ricerca di Sistema Elettrico

Accordo di Programma Ministero dello Sviluppo Economico - ENEA

Area: Governo, Gestione e Sviluppo, del Sistema Elettrico Nazionale

Progetto: Nuovo Nucleare da Fissione: Collaborazioni Internazionali e sviluppo Competenze in Materia Nucleare

Responsabile del Progetto: Massimo Sepielli, ENEA



**CIRTEN**

**CONSORZIO INTERUNIVERSITARIO  
PER LA RICERCA TECNOLOGICA NUCLEARE**

**UNIVERSITA' DI PALERMO  
DIPARTIMENTO dell' ENERGIA  
Sezione INGEGNERIA NUCLEARE**

## **Prosecuzione della Partecipazione a Comitati e Gruppi Internazionali**

F. Mascari, G. Vella

**CIRTEN-UNIPA RL- PAR 2011 1215/1250**

**Palermo, Agosto 2012**

*Lavoro svolto in esecuzione della linea progettuale LP1.C2 dell'AdP ENEA MSE,  
"Nuovo Nucleare da Fissione"*

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POWER2012

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# **SOMMARIO**

Nell'ambito dell'attività relativa alla linea progettuale LP1.C2 "Prosecuzione della partecipazione a comitati e gruppi internazionali" dell'Adp ENEA MSE, di seguito è riportata una sintesi degli interventi, dei temi e dei contributi dal nostro gruppo di ricerca presentati al SPRING 2012 CAMP MEETING e alla conferenza internazionale ICONE20-POWER2012.

## 1 PARTECIPAZIONE AL PROGRAMMA DI RICERCA CAMP

Nell'anno 2012 nell'ambito delle attività promosse dal USNRC la sede di Palermo ha partecipato allo "SPRING 2012 CAMP MEETING".

Si sono recati a tale gruppo di lavoro il Prof. Giuseppe Vella e il Dr. Fulvio Mascari.

Il meeting si è tenuto dal 30 Maggio 2012 al 1 Giugno 2012 a Lubiana in Slovenia.

Il Code Applications and Maintenance Program (CAMP) è stato formato dalla USNRC e altri partner internazionali al fine di avere uno scambio di informazioni su problematiche termoidrauliche di sicurezza degli impianti nucleari. Il gruppo si incontra due volte l'anno. Il rappresentante CAMP per l'Italia è il Dr. Roberto Ranieri di ISPRA. Partecipando al CAMP si ha la possibilità di avere accesso ai codici di termoidraulica di sistema di tipo "Best Estimate" tipo TRACE, RELAP5 e di cinetica neutronica tridimensionale tipo PARCS necessari per le attività di ricerca da noi svolte.

L'agenda della riunione è riportata nella Appendice 1.

Gli argomenti di interesse trattati in questa riunione sono stati:

- Status of NRC Code Development
- Program and Code Status
- Member Country Reports
- Technical reports
- TPC Meeting

Di particolare interesse per le nostre attività di ricerca è il prossimo rilascio della patch 3 del codice TRACE.

Nella riunione sono stati da noi presentati tre *technical report*:

- Analyses of the OSU-MASLWR natural circulation phenomena by using TRACE code.  
*Autori: Fulvio Mascari, Giuseppe Vella, Brian G. Woods, Kent Welter, Francesco D'Auria.*
- Analyses of the SPES-3 accident condition by using TRACE code.  
*Autori: Fulvio Mascari, Giuseppe Vella*
- Analyses of the TRACE-PARCS coupling capability.  
*Autori: Fulvio Mascari, Giuseppe Vella, Flavio Parozzi, Vincenzo Casamassima*

**Il primo technical report** si basa sulla attività realizzata presso il Dipartimento dell'Energia in collaborazione con Oregon State University, NuScale e l'Università degli studi di Pisa, relativa all'impianto OSU-MASLWR in operazione presso Oregon State University (OSU).

L'impianto sperimentale OSU-MASLWR è stato progettato per investigare il comportamento termoidraulico del reattore nucleare MASLWR, prototipo di un reattore avanzato integrale a generatore di vapore elicoidali in cui il fluido primario, in circolazione naturale, asporta la potenza prodotta nel core in condizioni normali di funzionamento.

Una nodalizzazione TRACE dell'impianto OSU-MASLWR è stata sviluppata presso il Dipartimento dell'Energia dell'Università degli Studi di Palermo in collaborazione con Oregon State University. Una campagna sperimentale è stata condotta al fine di caratterizzare termoidraulicamente i fenomeni tipici del reattore MASLWR.

I transitori studiati sono il test OSU-MASLWR-001, che consiste in un "*inadvertent actuation of 1 submerged ADS valve*" e ha lo scopo di riprodurre, sperimentalmente, la conseguente depressurizzazione del sistema primario e il conseguente comportamento del contenimento ad esso accoppiato, e il test OSU-MASLWR-002, "*natural circulation at core power up to 210 kW*", che colleziona sperimentalmente le portate volumetriche nel circuito primario e il grado di surriscaldamento all'uscita dei tubi elicoidali al variare della potenza fornita al core e della portata di FW.

I risultati presentati mostrano che i fenomeni che caratterizzano il test OSU-MASLWR-002 sono qualitativamente predetti dal codice nelle differenti condizioni di potenza del core e di portata secondaria. L'analisi del test OSU-MASLWR-001 mostra un generale accordo qualitativo con i dati sperimentali. Il codice TRACE è capace di predire i fenomeni tipici del transitorio quali lo svuotamento del circuito primario, la fase di "refill" del circuito primario e la fase di "long term cooling" tipica del MASLWR.

**Il secondo technical report** si basa sulla attività realizzata presso il Dipartimento dell'Energia in collaborazione con ENEA e SIET relativa all'impianto SPES-3. Tale impianto, è in fase di realizzazione presso la SIET, per caratterizzare i fenomeni termoidraulici che si destano, in transitori seguenti a postulati eventi incidentali nel reattore IRIS.

Una nodalizzazione TRACE dell'impianto SPES-3 è stata sviluppata presso il Dipartimento dell'Energia dell'Università degli studi di Palermo al fine di condurre analisi di eventuale supporto al progetto dell'impianto sperimentale SPES-3 e, dopo la costruzione dello stesso, calcoli di pre-test e calcoli di post-test.

In questo lavoro sono presentati i risultati di analisi di un caso di "low elevation SBLOCA" dovuto ad una rottura a ghigliottina della linea di iniezione diretta nel vessel. Tali analisi sono state paragonate con le analisi RELAP5 sviluppate dalla SIET.

Il paragone con i dati di simulazioni RELAP5, sviluppati dalla SIET, mostra un buon accordo in relazione ai più importanti parametri termoidraulici caratterizzanti il transitorio. In particolare l'uso del componente tridimensionale vessel, disponibile in TRACE, permette una più dettagliata analisi dei fenomeni attesi. Questo, ovviamente, implica un incremento del tempo di calcolo.

**Il terzo technical report** si basa sulla attività realizzata presso il Dipartimento dell'Energia in collaborazione con RSE relativa all'uso di codici termoidraulici di sistema e neutronici tridimensionali accoppiati.

Storicamente al fine di produrre analisi aventi in oggetto il progetto, l'ottimizzazione e la sicurezza di impianti nucleari, sono stati sviluppati software dedicati (codici di calcolo) miranti alla studio di particolari problematiche. Nell'ambito degli studi di analisi di sicurezza degli impianti nucleari hanno trovato largo impiego codici di simulazione termoidraulica e neutronica il cui utilizzo

permette un miglioramento nella sicurezza degli impianti di potenza già esistenti, una migliore utilizzazione del combustibile nucleare, un aumento della flessibilità operativa. Con tali codici è inoltre possibile realizzare studi miranti all'estensione del funzionamento dei reattori, allo sviluppo di procedure di emergenza, allo studio di eventi operazionali ed infine alla realizzazione di simulatori per l'addestramento degli operatori. In questo contesto al fine di modellare le interazioni tra fenomeni di diversa natura, ma intimamente correlati, presenti in un reattore nucleare, sono state sviluppate tecniche di accoppiamento tra codici di calcolo aventi differenti campi di applicazione. Le analisi accoppiate hanno la potenzialità di produrre più accurati studi, ridurre le eventuali incertezze e dunque fornire una più realistica analisi dei fenomeni in oggetto che porta a una più completa e migliore comprensione delle sequenze dovute a postulati eventi incidentali.

Al fine di qualificare l'uso di procedure di accoppiamento di singoli codici "best estimate" e valutare le loro capacità, nell'ultimo decennio sono stati sponsorizzati dall'OECD/NEA tre benchmark internazionali:

- PWR Main Steam Line Break in TMI-1
- BWR Turbine Trip in Peach Bottom
- VVER1000 coolant transient.

Allo scopo di verificare le capacità dei codici accoppiati TRACE/PARCS viene presentato lo studio del transitorio di riferimento "PWR Main Steam Line Break" (MSLB) già proposto dal NEA, agenzia dell'OECD, e promosso dall'USNRC. Il benchmark è basato sui dati di progetto di un reale PWR della Babcock and Wilcox e sui dati operativi dell'impianto Three Miles Island 1.

Tra gli scopi principali di tale benchmark si ricordano quelli di verificare le capacità dei codici di analizzare transitori complessi con accoppiate interazioni neutroniche e termoidrauliche in condizioni tridimensionali e quelli di saggiare gli accoppiamenti di codici neutronici e termoidraulici.

I risultati delle analisi, ottenute simulando un "main steam line break" in un tipico PWR della Babcock and Wilcox, mostrano che analisi accoppiate hanno la capacità di riprodurre i fenomeni tipici di un reattore nucleare e la loro interazione. Le capacità grafiche di SNAP sono avanzate e utili per l'analisi e visualizzazione dei dati prodotti rendendo possibili visualizzazioni bidimensionali e tridimensionali.



## 2 PARTECIPAZIONE ALLA CONFERENZA INTERNAZIONALE ICONE20-POWER2012

La conferenza a cui si è partecipato è la “2012 20th International Conference on Nuclear Engineering and the ASME 2012 Power Conference (ICONE20-POWER2012).

Si è recato a tale conferenza il Dr. Fulvio Mascari.

La conferenza si è tenuta dal 30 Luglio 2012 al 3 Agosto 2012 ad Anaheim, California, USA.

ICONE20-POWER2012 è una Conferenza Internazionale organizzata da Nuclear and Power Divisions of the American Society of Mechanical Engineers (ASME), la Japan Society of Mechanical Engineers (JSME) e la Chinese Nuclear Society (CNS) in cooperazione con l'International Atomic Energy Agency (IAEA), la Canadian Nuclear Society (CNS), l'European Nuclear Society (ENS), la Society Nuclear Mexicana (SNM), la Nuclear Society of Slovenia (NSS), l'Atomic Energy Society of Japan (AESJ), l'Austrian Nuclear Society (ÖKTG) e la Canadian Standard Association (CSA). Lo scopo di questa conferenza è di mantenere i contatti tra gli esperti nelle aree dell'ingegneria nucleare, della termoidraulica e dell'energie rinnovabili mantenendoli aggiornati sugli ultimi sviluppi in questi settori.

Le varie *"technical track"* della conferenza con i vari contributi sono riportati nella appendice 2. Nella appendice 3 è riportato il programma del convegno. In particolare sono state presentati contributi su:

- Plant Operations, Maintenance, Engineering, Modifications, Life Cycle, and Balance of Plant;
- Component Reliability and Materials Issues;
- Plant Systems, Structures, and Components;
- Steam Generator Technology Applications and Innovations;
- Advanced Reactors and Near-Term Deployment;
- Safety and Security;
- Codes, Standards, Licensing, and Regulatory Issues;
- Fuel Cycle, Radioactive Waste Management and Decommissioning;
- Thermal-Hydraulics;
- Computational Fluid Dynamics (CFD) and Coupled Codes;
- Instrumentation and Controls;
- Next Generation Systems;
- Fusion Engineering;
- Reactor Physics, Neutronics, and Transport Theory;
- Nuclear Education, Human Resources, and Public Acceptance;
- Design Basis and Beyond Design Basis Events;
- Fuels and Combustion, Materials Handling, Emissions;
- Heat Exchangers and Cooling Systems;
- Turbines, Generators, and Auxiliaries;

- Advanced Energy Systems and Renewables (Wind, Solar, Geothermal);
- Performance Testing and Performance Test Codes;
- Simple and Combined Cycles.

Durante il convegno è stata organizzata anche una competizione tra studenti (Student Paper Competition).

Nella riunione è stato da noi presentato un articolo dal titolo:

"TRACE AND RELAP5 CODES FOR BEYOND DESIGN ACCIDENT CONDITION SIMULATION IN THE SPES3 FACILITY"

i cui autori sono in ordine, Roberta Ferri, Fulvio Mascari, Paride Meloni e Giuseppe Vella.

Tale articolo si basa sull'attività realizzata presso il Dipartimento dell'Energia in collaborazione con l'ENEA e la SIET relativa all'impianto SPES-3. Questo impianto è in fase di realizzazione presso la SIET, per caratterizzare i fenomeni termoidraulici che si destano, in transitori seguenti a postulati eventi incidentali nel reattore IRIS, indagando allo stesso tempo il comportamento dei sistemi passivi considerati in sede di progetto e le interazioni fra i vari componenti del contenimento e il primario. I dati sperimentali prodotti nell'impianto possono essere utilizzati anche per la validazioni di codici termoidraulici di sistema. I risultati di tale impianto potranno comunque essere di validità per i reattori nucleari integrati in generale.

Una nodalizzazione TRACE dell'impianto SPES-3 è stata sviluppata presso il Dipartimento dell'Energia dell'Università degli studi di Palermo al fine di condurre analisi di eventuale supporto al progetto dell'impianto sperimentale SPES-3 e, dopo la costruzione dello stesso, calcoli di pre-test, per contribuire al progetto termoidraulico dei test di interesse, e calcoli di post-test, con lo scopo di validare il codice TRACE.

In questo lavoro sono presentati i risultati delle analisi di un caso di "DVI line DEG break "in design and beyond design conditions dovuto ad una rottura a ghigliottina della linea di iniezione diretta nel vessel. Tali analisi sono state paragonate con le analisi RELAP5 sviluppate dalla SIET.

Il paragone con i dati delle simulazioni RELAP5, mostra un buon accordo in relazione ai più importanti parametri termoidraulici caratterizzanti il transitorio in design and beyond design condition. In particolare l'uso del componente tridimensionale vessel, disponibile in TRACE, nella modellazione della Reacor Cavity, del Pressure Suppression System, del Dry Well e del RWST permette una più dettagliata analisi dei fenomeni attesi senza ricorrere all'accoppiamento di codici di sistema e specifici codici di contenimento. Questo, ovviamente, implica un incremento del tempo di calcolo.

Il Dr. Fulvio Mascari ha anche partecipato ai lavori del "Nuclear Engineering Division Technical Committee Meetings on Thermal Hydraulics".

## APPENDICE 1

All Technical Sessions are at City Hotel conference center

Wednesday, May 30, 2012

8:15 AM Registration

8:30 AM Opening and Welcome

Leon Cizelj, Jožef Stefan Institute, Reactor Engineering Division, RED Head  
Franci Demšar, Slovene Research Agency, Director  
Stane Rožman, Nuclear Power Plant Krško, Director  
Andreja Peršič, Slovenian Nuclear Safety Administration, Nuclear Safety Division  
Borut Mavko and Andrej Prošek, Jožef Stefan Institute, Reactor Engineering Division

8:50 AM Chris Hoxie, USNRC, USA

[Status of NRC Code Development \(Chris L. Hoxie\)](#)

### **Program and Code Status**

**Chair: Antony Calvo, USNRC**

9:20 AM Antony Calvo, USNRC, USA

Review and Approval of the Fall 2011 Meeting Minutes

9:30 AM Douglas Barber, ISL, USA

[RELAP5 Status and User Problem Report \(Douglas Barber, Dan Prelewicz\)](#)

10:00 AM Break

10:15 AM Josh Whitman, USNRC, USA

[TRACE Code Development Status \(Joshua Whitman\)](#)

### **Member Country Reports**

**Chair: Antony Calvo, USNRC**

10:45 AM Chiung-Wen Tsai, National Tsing Hua University, Taiwan

[Status of CAMP Activities in Taiwan \(Jong-Rong Wang, Chiung-Wen Tsai, Hao-Tzu Lin, Chunkuan Shih\)](#)

11:15 AM Jozsef Banati, Chalmers University of Technology, Sweden

[Summary of Swedish CAMP Related Activities \(Alexander Agung, József Bánáti, Ninos Garis, Weimin Ma\)](#)

11:45 AM Ernest Staroń, National Atomic Energy Agency, Poland

[The Polish Regulatory Body Activities \(Ernest Staroń, Marcin Dąbrowski\)](#)

12:15 PM Lunch

1:15 PM Miguel Sánchez Perea, Nuclear Safety Council, Spain

[Status Report of CAMP Activities in Spain \(Miguel Sánchez Perea\)](#)

1:45 PM Omar Zerkak, Paul Scherrer Institut, Switzerland

[Selected PSI activities using CAMP Agreement codes \(C. Adamsson, J. Freixa, S. Canepa, T. Gudmundsson, D. Karanki, T-W. Kim, K. Nikitin, D. Papini, R. Szijarto\)](#)

2:15 PM Seunghoon Ahn, Korea Institute of Nuclear Safety, Korea

[Status of CAMP Activities in Korea \(Seung-Hoon Ahn\)](#)

2:45 PM Davor Grgić, University of Zagreb, Croatia

[Status of CAMP Activities in Croatia \(Vesna Benčik, Davor Grgić, Siniša Šadek\)](#)

3:15 PM Break

3:30 PM Jinjun Feng, Nuclear and radiation Safety Center, China

[CAMP Activities Status in China \(Feng Jinjun, Chai Guohan\)](#)

4:00 PM Aleksandar Delja, Canadian Nuclear Safety Commission, Canada

Canada CAMP (Code Applications and Maintenance Program) Updates (Aleksander Delja)

**Technical Reports**

**Chair: Iztok Tiselj, JSI**

4:30 PM Wadim Jaeger, Karlsruhe Institute of Technology, Germany

[Uncertainty and Sensitivity Study with TRACE-DAKOTA and TRACE-SUSA: A comparison based on NUPEC BFBT experimental data \(Wadim Jäger, Victor Sánchez, Francisco Montero, Cesar Queral\)](#)

5:00 PM Ovidiu-Adrian Berar, Jožef Stefan Institute, Slovenia

[IJS conversion procedure from RELAP5 to TRACE - example of Achilles test rig \(Ovidiu-Adrian Berar, Andrej Prošek, Borut Mavko\)](#)  
[Animation](#)

5:30 PM Adjourn

7:30 PM Welcome Reception, City hotel Ljubljana (sponsored by Nuclear Power Plant Krško)

Thursday, May 31, 2012

8:00 AM	<b><u>Technical Reports (continued)</u></b>	<b><u>Chair: Josh Whitman, USNRC</u></b>
8:00 AM	Pavel Kral, Nuclear Research Institute Řež, Czech Republic	<a href="#"><u>Application of RELAP5 and Uncertainty Methodology to Safety Analysis of VVER-1000 (Pavel Kral, Jelena Krhounkova, Frantisek Lahovsky, Jiri Macek, Radim Meca)</u></a>
8:30 AM	Fulvio Mascari, Università degli Studi di Palermo, Italy	<a href="#"><u>Analyses of the OSU-MASLWR natural circulation phenomena by using TRACE code (Fulvio Mascari, Giuseppe Vella, Brian G. Woods, Kent Welter, Francesco D'Auria)</u></a>
9:00 AM	Hideo Konishi, JNES, Japan	<a href="#"><u>Fukushima Daiichi Accident Analyses with RELAP5 (Hideo Konishi)</u></a>
9:30 AM	Tong Soo Choi, KEPCO Nuclear Fuel, Korea	<a href="#"><u>New Model and Correlation Package for RELAP5 Reflood Calculations (Tong Soo Choi)</u></a>
10:00 AM	Break	
10:15 AM	Masahiro Furuya, Central Research Institute of Electric Power Industry, Japan	<a href="#"><u>Code Validation for Flashing-Induced Density Wave Oscillations in SIRIUS-N Facility Which Simulates ESBWR (Masahiro Furuya, Yoshihisa Nishi, Nobuyuki Ueda)</u></a>
10:45 AM	Filippo Fiori, GRNSPG, Italy	<a href="#"><u>Status of the activity with RELAP5 and TRACE at GRNSPG (A. Petruzzi, M. Cherubini, W. Giannotti, N. Muellner, A. Kovtonyuk, F. Fiori, E. Coscarelli)</u></a>
11:15 AM	Kiril Velkov, GRS, Germany	<a href="#"><u>First Applications of PARCS and SNAP at GRS mbH (Kiril Velkov, Yann Perin, Carsten Köllein)</u></a>
11:45 AM	Petr Heralecký, TES, Czech Republic	<a href="#"><u>Post-Test Analysis of Hot Leg 2x25% Break at PSB VVER Facility Using RELAP5/MOD3.3 Code (Petr Heralecký)</u></a>
12:15 PM	Lunch	
1:15 PM	Jose Felipe Villanueva, Universitat Politècnica de València, Spain	<a href="#"><u>Spent Fuel Pool Analysis Using TRACE Code (Francisco Sánchez, Sofia Carlos, Jose Felipe Villanueva, Sebastián Martorell)</u></a>
1:45 PM	Fulvio Mascari, Università degli Studi di Palermo, Italy	<a href="#"><u>Analyses of the SPES-3 accident condition by using TRACE code (Fulvio Mascari, Giuseppe Vella)</u></a>
2:15 PM	César Queral, Universidad Politecnica de Madrid, Spain	<a href="#"><u>Analysis of LOCA sequences with TRACE code (César Queral, Juan Gonzalez-Cadelo, Javier Montero)</u></a>
2:45 PM	Viktor Mukoid, State Scientific and Technical Center for Nuclear and Radiation Safety, Ukraine	<a href="#"><u>Development of the TRACE Model for WWER-1000 Reactor Type (Yuriy Alekseev, Valeriy Shikhabutinov, Viktor Mukoid)</u></a>
3:15 PM	Break	
3:30 PM	Benoit Arsenault, AMEC NSS, Canada	<a href="#"><u>Benchmarking of a Generic CANDU Reactor with PARCS, MCNP and RFSP-IST (Benoit Arsenault, Omar Shaikh, David Luxat, Dan Jabaay, Yunlin Xu, Andrew Ward, Thomas</u></a>

[Downar\)](#)

4:00 PM Sergio Gallardo, Universidad  
Politecnica de Valencia, Spain

[PKL/ROSA Counterpart Test. Post Test analysis with TRACE5  
\(S. Gallardo, A. Querol, G. Verdú\)](#)

4:30 PM Gumersindo Verdú, Universitat  
Politècnica de València, Spain

[IMPROVEMENTS IN THE DECAY HEAT MODEL IN THE  
THERMALHYDRAULIC CODE TRAC-BF1 \(A. Soler, T.  
Barrachina, R. Miró, G. Verdú, A. Concejal, J. Melara\)](#)

5:30 PM Ljubljana Tour (1.5 hour)

8:00 PM Social dinner, City hotel Ljubljana (sponsored by GEN energija)

Friday, June 1, 2012

**Technical Reports (continued)**

**Chair: Andrej Prošek, JSI**

- 8:00 AM César Queral, Universidad Politecnica de Madrid, Spain [Simulation of CCW and SGTR sequences in a PWR-W \(César Queral, Luisa Ibáñez, Gonzalo Jiménez-Varas\)](#)
- 8:30 AM Fulvio Mascari, Università degli Studi di Palermo, Italy [Analyses of the TRACE–PARCS coupling capability \(Fulvio Mascari, Giuseppe Vella, Flavio Parozzi, Vincenzo Casamassima\)](#)  
[Animations](#)
- 9:00 AM Rafael Miró, Polytechnic University of Valencia, Spain [A COUPLING PROCEDURE BETWEEN TRAC-BF1 AND PARCS CODES FOR 3D TRANSIENT SIMULATIONS \(A. Jambrina, T. Barrachina, R. Miró, G. Verdú, A. Concejal, J. Melara\)](#)
- 9:30 AM Andrej Prošek, Jožef Stefan Institute, Slovenia [RELAP5, TRACE 1D and TRACE 3D comparison against Bethsy 9.1b test \(Andrej Prošek, Ovidiu-Adrian Berar\)](#)
- 10:00 AM Break
- 10:15 AM Suren Bznuni, Nuclear and Radiation Safety Centre, Armenia [Experience of PARCS-RELAP Implementation for WWER reactors modelling \(S. Bznuni, T. Malakyan, A. Amirjanyan\)](#)

**TPC Meeting**

**Chair: Antony Calvo, USNRC**

- 10:45 AM Acceptance of Minutes from Fall 2011 TPC Meeting
- 10:45 AM [NUREG/IA and CAMP Status \(Antony Calvo\)](#)
- 11:15 AM Discussion of In-Kind Contributions
- 11:45 PM Final Discussion
- 12:30 PM End of Spring 2012 CAMP Meeting

## APPENDICE 2

- **Plant Operations, Maintenance, Engineering, Modifications, Life Cycle, and Balance of Plant**

**ICONE20-POWER2012-54027**

Improving Nuclear Power Plant's Operational Efficiencies in the USA

*Joseph S. Miller, Bob Stakenborghs, and Robert W. Tsai*

**ICONE20-POWER2012-54092**

Developing the Initial Reliability Assurance Program (RAP) for Maintenance in New Nuclear Plants

*J. K. August and Edward Dundon*

**ICONE20-POWER2012-54094**

One Power Plant's Experience in Setting Up, Performing and Acting on a Pipe Hanger Surveillance Program

*Lange Kimball and Kuda Mutama*

**ICONE20-POWER2012-54109**

Development of a Manipulator-Supported Laser Decontamination System

*Wolfgang Lippmann, Rene Littwin, and Antonio Hurtado*

**ICONE20-POWER2012-54158**

Insights From Investigations of High Energy Arcing Fault "HEAF" Events in German Nuclear Power Plants

*Marina Röwekamp, Steffen Katzer, Joachim Klindt, and Heinz-Peter Berg*

**ICONE20-POWER2012-54165**

Effects of German Nuclear Power Plants Closure on Future Electricity Cost in Central Europe

*Miroslav Vitek*

**ICONE20-POWER2012-54189**

A Method for Incorporating Thermal Performance Characterization Tools With Real-Time and Historic Plant Data

*Michael L. Morgan, Jr.*

**ICONE20-POWER2012-54276**

Price Evaluation in Nuclear Power Industry: Modeling and Computation

*Vasyl O. Kostyuk, Taras O. Kostyuk, and Volodymyr K. Dobrovolsky*

**ICONE20-POWER2012-54290**

Evaluation of Reactor Recirculation Pump Adjustable Speed Drive Cell Bypass Recovery Scenarios for the Exelon Nuclear Plants

*James W. Morgan, Hilary A. Neal, Eric R. Frantz, Mark Budelier, Hossein Youssefnia, and Thomas K. Lindqvist*

**ICONE20-POWER2012-54329**

A Study for Heat-Loss Characteristics of Hot-Water Layer by the Increment of Reactor Power

*Young-Chul Park, Kyoung-Woo Seo, Dae-Young Chi, Ju-Hyeon Yoon, and Hyun-Gi Yoon*

**ICONE20-POWER2012-54348**

Optimizing Capacity of Nuclear Power Plants: Thermal Performance Assessment Using a Case Study

*Komandur Sunder Raj*



**ICONE20-POWER2012-54352**

Developing a Thermal Performance Monitoring System Specification for a Nuclear Power Plant: A Case Study

*Komandur Sunder Raj*

**ICONE20-POWER2012-54406**

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- **Turbines, Generators, and Auxiliaries**

**ICONE20-POWER2012-54330**

Research on Low Cycle Fatigue in 26m<sup>2</sup> Low Pressure Inner-Casing of Nuclear Power Plant

*Yifeng Hu, Gang Chen, and Ming Kang*

**ICONE20-POWER2012-54434**

Missile Safety Analysis of Nuclear Steam Turbine

*Rong Chen, Feng Liang, Wen Xiang Hua, Pu Ning Jiang, and Xiao Zhong He*

**ICONE20-POWER2012-54555**

Operation and Maintenance Solutions for Generator Water Cooling

*Matthias Svoboda and Robert Svoboda*

**ICONE20-POWER2012-54584**

Failure of Modern Generators

*Clyde V. Maughan*

**ICONE20-POWER2012-54694**

The Advantages of Using Hydraulic Coupling Bolts Over Standard Fitted Bolts in Steam Turbine Flange Couplings

*Tom Kennedy*

**ICONE20-POWER2012-54779**

Lifetime Assessments of an Old Westinghouse-Design Steam Chest

*Sazzadur Rahman, Waheed Abbasi, and Thomas W. Joyce*

**ICONE20-POWER2012-54805**

A Compressible Three-Dimensional Inverse Design Method Based on the Streamline Curvature Approach and Clebsch Formulation for Radial and Mixed Flow Turbomachines

*Xiao Pei Tian and Peng Shan*

**ICONE20-POWER2012-54862**

Steady and Unsteady Flow Measurements Under Low Load Conditions in a Low Pressure Model Steam Turbine

*Kiyoshi Segawa, Shigeki Senoo, Hisashi Hamatake, Takeshi Kudo, Tateki Nakamura, and Naoaki Shibashita*

**ICONE20-POWER2012-54887**

Steam Turbine Overspeed Protection Failures, Causes and Strategies to Avoid Them

*Wallace F. Ebner*

**ICONE20-POWER2012-55029**

Subsynchronous Resonance and Torsional Effects on a Steam Turbine Generator in Transmission Systems With Series Capacitor Compensation

*K. R. Mutama, D. H. Baker, R. D'Aquila, B. Fitzgerald, C. Wegner, R. M. Staulters, and J. Seeliger*

**ICONE20-POWER2012-55093**

The Effects of Cycling on Generator Rotors

*Bill Moore*

**ICONE20-POWER2012-55139**

A Resurgence of Torsional Vibration Concerns for Nuclear and Fossil Steam Turbine Generator Retrofits

*Stephen R. Reid and James B. Lewis*

**ICONE20-POWER2012-55140**

Proven Methods to Repair and Extend the Life of Low Pressure Turbine Rotors

*Stephen R. Reid and James B. Lewis*

**ICONE20-POWER2012-55208**

Main Generator Life Cycle Management

*Arun Puri and John DiBiase*

**ICONE20-POWER2012-55223**

Steam Turbine, Boiler, and Valve Optimization Strategies to Recover Parasitic Load and Provide Life Extension

*Michael W. Smiarowski*

- **Advanced Energy Systems and Renewables (Wind, Solar, Geothermal)**

**ICONE20-POWER2012-54216**

Basic Characteristics of a Compact PV/T Simulator

*Elkata Yandri, Naoto Hagino, Kazutaka Itako, and Hiro Yoshida*

**ICONE20-POWER2012-54424**

Thermodynamic and Economic Analysis of Reheat Transcritical Organic Rankine Power Cycle Using a Low Temperature Geothermal Heat Source

*Hanfei Tuo*

**ICONE20-POWER2012-54532**

Evaluation of Distributed Generation Technologies in a University Context

*Marco Gambini and Michela Vellini*

**ICONE20-POWER2012-54576**

A CO<sub>2</sub> Compression and Dehydration System Utilizing Absorption Chillers and Heat Recovery Concepts

*David Hasler, David Stopek, Roger Smith, and John Klumphyan*

**ICONE20-POWER2012-54799**

Process Simulation for Enhanced Energy Recovery in a 10 MW(e) Base-Load Solar Thermal Power Plant

*Sadaf Siddiq, Zafar Ullah Koreshi, and Shahab Khushnood*

**ICONE20-POWER2012-54811**

Parameter Analysis of Low Grade Thermal Energy Sources Use With Organic Rankine Cycle

*Gang Zhao, Jie Wang, and Qian Shi*

**ICONE20-POWER2012-55115**

A Highly Efficient Cogeneration System Using APT Coupled With Biomass Gasification

*Y. Tsujikawa*

**ICONE20-POWER2012-55120**

Rooftop Solar Electric Generator Project

*Jack Robinson, Jr. and Robert J. Durscher*

**ICONE20-POWER2012-55151**

Economical Impact of Using Wind/PV Hybrid Systems on Energy Sector in Jordan

*Mohammad AL Zubi, Trilochan Singh, and Hesham AL Salem*

**ICONE20-POWER2012-55224**

A Design of a Hydrofoil Family for Current-Driven Marine-Hydrokinetic Turbines

*Henry Shiu, C. P. "Case" van Dam, Matthew Barone, Erick Johnson, Ryan Phillips, William Straka, Arnold Fontaine, and Michael Jonson*

- **Performance Testing and Performance Test Codes**

**ICONE20-POWER2012-54136**

Experimental Investigations of Control Rod Drive Mechanism

*Wenyuan Xiang, Yonghong Lv, Wenyong Huang, Gguangyao Lu, and Jianming Zhou*

**ICONE20-POWER2012-54551**

Combined Cycle Phased Testing Philosophy: Revisited

*Jagadish Nanjappa and Mike Gross*

**ICONE20-POWER2012-54609**

Practical Considerations for Power Plant Thermal Performance Test Accuracy

*Evan E. Daigle, Thomas P. Schmitt, and Christopher R. Banares*

**ICONE20-POWER2012-54971**

Efficiency Based on Free-Energy Instead of Isentropic Conditions

*William Ernest Schenewerk*

**ICONE20-POWER2012-55039**

Total Solar Field Direct Normal Insolation Measurement Method Under Investigation for ASME PTC 52

*Dave W. Price, Keith Kirkpatrick, Wayne D. Ferguson, and Dudley Benton*

**ICONE20-POWER2012-55102**

Performance Testing of a Multi-Stage Centrifugal Compressor With Sidestream Extraction

*A. I. C. Hunter, R. G. Nyquist, M. F. Andrews, J. Smith, and H. Boice*

**ICONE20-POWER2012-55212**

A Study of Venturi Tubes

*Richard L. Wakeland*

- **Simple and Combined Cycles**

**ICONE20-POWER2012-54711**

How the Use of Limited Plant Data Can Support Limited Improvements in Combined Cycle Operation

*James M. Perez and Tina L. Toburen*

**ICONE20-POWER2012-54719**

Current Technology for Power Plant Makeup Water Treatment and Wastewater Recovery

*Brad Buecker*

**ICONE20-POWER2012-54822**

Performance Evaluation of Evaporative Compressor Inlet Air Cooling System in a Gas Turbine-Based Cogeneration Plant

*Farshid Zabihian, Alan S. Fung, and Fabio Schuler*

**ICONE20-POWER2012-54902**

The Convergence of Squeeze: With Respectable Speed, a New Gas Turbine Power Plant Rises at an Arizona Mine

*John Baker and Marshall Ralph*

**ICONE20-POWER2012-55111**

Smart Lube Systems for Gas Turbine Engines

*Thomas B. Kenney*

**ICONE20-POWER2012-55192**

Numerical Study on Flow Separation Control for High-Lift Low-Pressure Turbine Split Blade

*Jianhui Chen, Yonghui Xie, Di Zhang, and Zhongyang Shen*



## APPENDICE 3

### ICONE20 POWER2012 PROGRAM

	Room Size	Tower	Monday, July 30th		Tuesday, July 31st					Wednesday, August 1st					Thursday, August 2nd					Friday, August 3rd							
TIMES	POWER	ICONE	8:00am - 12:00pm	8:00am - 5:00pm	8:00am - 10:00am	10:30am - 12:00pm	12:15am - 1:15pm	1:30pm - 3:15pm	3:45pm - 5:30pm	8:00am - 9:45am	10:15am - 12:00pm	12:15am - 1:15pm	1:30pm - 3:15pm	3:45pm - 5:30pm	6:30pm - 8:30pm	8:00am - 9:45am	10:15am - 12:00pm	12:15am - 1:15pm	1:30pm - 3:15pm	3:45pm - 5:30pm	5:40am - 6:40pm	8:00am - 9:45am	10:15am - 12:00pm	1:00pm - 4:00pm			
Exhibit Hall B	80 ppl	Lower					Lunch	21-2	21-9		21-1	Lunch	21-4	21-3			21-8	Lunch	21-7	21-6		21-5					
Exhibit Hall C	60 ppl	Lower						1-15	1-14	1-3	1-4			1-5	1-6		9-18		9-19		9-21	9-21		7-8			
Exhibit Hall D	60 ppl	Lower						12-1	12-2	12-3	12-4			12-5	11-3		9-24		9-34		9-25	9-35			8-2	8-6	
Exhibit Hall E	60 ppl	Lower						4-9	4-10	2-8	5-1			5-2	10-10		1-7		1-12		10-7	10-8			17-5	17-6	
Exhibit Hall G	60 ppl	Lower						20-1	20-8	5-3	20-3			20-4	20-2		2-15		20-5		20-6	9-28					
Exhibit Hall H	60 ppl	Lower						2-1	2-2	2-13	2-6			2-17	2-14		2-16		2-5		18-14	18-5		2-4	2-7		
Exhibit Hall I	60 ppl	Lower						19-1	11-1	11-2	4-11			19-3	19-4		7-6		19-5					19-9			
Exhibit Hall J	60 ppl	Lower					7-1	7-2	7-3	7-4		17-1	7-5		17-2	17-3		17-4	7-7			6-15	6-16				
North Ballroom A/B	200 ppl	Lower				FOOD	FOOD		FOOD	FOOD					ICONE AWARDS Reception							ICONE Students AWARDS Reception			Steering Committee Exit Meeting		
Center Ballroom	720 ppl	Lower			Opening Keynote I	Keynote II		ICONE Plenary I		Power Plenary I						Power Plenary II	ICONE Plenary II		ICONE Plenary III	18-18							
South Ballroom A	200 ppl	Lower					Lunch		Comm Mtg		18-11	Lunch	18-13	18-17							18-16						
South Ballroom B	200 ppl	Lower							Comm Mtg		18-15			18-6	18-12												
Magic Kingdom 1	250 ppl	Upper		Workshop or Tutorial					9-1	9-26	9-27		9-3		9-4												
Magic Kingdom 2	300 ppl	Upper		Workshop or Tutorial					9-6	9-29	9-30		9-31		9-11												
Magic Kingdom 3	300 ppl	Upper		Workshop or Tutorial				9-8	9-9	9-10	9-16		9-32														
Magic Kingdom 4	250 ppl	Upper		Workshop or Tutorial				10-1	10-2	10-3	10-4		10-5														
A Ticket	25 ppl	Upper						Practice																			
B Ticket	100 ppl	Upper						22-1	4-12																		
Sleeping Beauty	200 ppl	Upper		Comm Mtg	ICONE Steering Committee Initial Meeting 12:00 1:30 PM	Comm Mtg	Comm Mtg	NED TC # 1 Meeting	Comm Mtg	Comm Mtg	Comm Mtg	Comm Mtg	NED TC # 2 Meeting	Power Awards	Power Awards												
Monorail A	49 ppl	L Lower					Lunch	24-2	24-3	15-1	15-2	Lunch	24-1	24-7			15-3	1-1		6-11	6-14						
Monorail B	49 ppl	L Lower		Workshop or Tutorial					6-2	6-4	6-5		22-2		22-3	22-4			17-7			15-4					
Monorail C	49 ppl	L Lower		Workshop or Tutorial					3-2	3-3	3-1		3-4		6-6	6-7			6-10	3-5		3-6	3-7				
Castle A	49 ppl	L Lower		ASME	ASME	ASME		ASME	ASME	ASME	ASME		ASME		ASME	ASME			ASME	ASME		ASME	ASME	ASME	ASME	ASME	
Castle B	49 ppl	L Lower		Workshop or Tutorial					13-1	13-2	8-3		23-1		23-2	23-3											
Castle C	49 ppl	L Lower		Workshop or Tutorial				14-1	14-2	14-3							14-4			8-1	8-5						
Congo	30 ppl	Adventure					NED TC # 5 Meeting					NED TC # 7 Meeting										NED TC # 3 Meeting					
Zambezi	18 ppl	Adventure					NED EC Daily Meetings																				
Mark Twain	100 ppl	Frontier						18-9																			
Mississippi	42 ppl	Frontier					Lunch	16-1	Poster	16-5	Poster	Lunch	16-9	Poster			16-13	Poster									
Columbia	44 ppl	Frontier							16-2	Poster	16-6		Poster		16-10	Poster				Poster							
Wilderness	60 ppl	Frontier							16-3	Poster	16-7		Poster		16-11	Poster			Comm Meeting	Comm Meeting	NED TC # 4 Meeting	Comm Meeting	Comm Meeting	Comm Meeting			
Western	60 ppl	Frontier						16-4	Poster	16-8	Poster		16-12	Poster			Comm Meeting	Comm Meeting									

## APPENDICE 4

Breve curriculum scientifico del gruppo di lavoro impegnato nell'attività

Il prof. Giuseppe Vella , Ordinario del Settore Scientifico Disciplinare ING-IND/19 Impianti Nucleari, è Responsabile del reattore nucleare di ricerca AGN 201 - COSTANZA dell'Università di Palermo. Ha coordinato diversi programmi di ricerca finanziati dal Ministero della Ricerca Scientifica e Tecnologica. E' stato Coordinatore nazionale di un progetto di ricerca di interesse nazionale PRIN-2007 finanziato dal MIUR. E' autore o coautore di più di 140 articoli pubblicati su riviste scientifiche nazionali ed internazionali e/o presentati a conferenze, congressi o simposi. E' anche coautore di diversi rapporti nell'ambito dei PAR trascorsi. L'attività scientifica del prof. Vella ha riguardato principalmente le seguenti tematiche: tubi di calore al sodio, problemi di ribagnamento di superfici ad elevata temperatura, efflussi critici bifase, analisi termoidrauliche relative alla sicurezza dei reattori nucleari a fissione, analisi neutroniche, termoidrauliche e termomeccaniche del mantello e di componenti ad alto flusso termico di un tipico reattore a fusione di tipo TOKAMAK. E' stato referee per la rivista "Fusion Engineering and Design" e "Nuclear Engineering and Design".

Fulvio Mascari, assegnista di ricerca del S.S.D. ING-IND/19, da circa 6 anni svolge attività di ricerca su problematiche termoidrauliche e nucleari connesse allo sviluppo dei reattori a fissione. Partecipa alle attività di validazione dei codici termoidraulici di sistema TRACE e RELAP5, collaborando con il Department of "Nuclear Engineering & Radiation Health Physics" della Oregon State University, NuScale, l'Università degli Studi di Pisa e ENEA. Recentemente ha preso parte alle attività di un International Collaborative Standard Problem (ICSP) on "Integral PWR Design Natural Circulation Flow Stability and Thermo-Hydraulic Coupling of Containment and Primary System during Accidents" promosso dall'IAEA.

I risultati delle sue attività di ricerca sono stati pubblicati su una ventina di memorie in riviste internazionali, capitoli di libro, in atti di congresso nazionali e internazionali e rapporti. E' anche coautore di diversi rapporti nell'ambito dei PAR trascorsi. E' stato referee per le riviste "Nuclear Engineering and Design" e "Science and Technology of Nuclear Installations".